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H Maier et al.

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#### **Deuterium retention in tungsten based materials for fusion applications**

H. Maier<sup>a\*</sup>, T. Schwarz Selinger<sup>a</sup>, R. Neu<sup>a,b</sup>, C. Garcia-Rosales<sup>c,d</sup>, M. Balden<sup>a</sup>, A. Calvo<sup>c,d</sup>, T. Dürbeck<sup>a</sup>, A. Manhard<sup>a</sup>, N. Ordás<sup>c,d</sup>, T. F. Silva<sup>a,e</sup>

<sup>a</sup>Max-Planck-Institut für Plasmaphysik, , Boltzmannstr. 2, 85748 Garching, Germany <sup>b</sup>Technische Universität München, Boltzmannstrasse 15, 85748 Garching, Germany <sup>c</sup>Ceit-IK4, Paseo de Manuel Lardizabal 15, 20018 San Sebastian, Spain <sup>d</sup>Universidad de Navarra, Tecnun, Paseo de Manuel Lardizabal 13, 20018 San Sebastian, Spain e Instituto de Física da Universidade de São Paulo, Rua do Matão, trav. R 187, 05508-090 São Paulo, Brazil

\*Corresponding author: Hans Maier Max-Planck-Institut für Plasmaphysik Boltzmannstr. 2 85748 Garching Germany

Email: [Hans.Maier@ipp.mpg.de](mailto:Hans.Maier@ipp.mpg.de) phone: +49 89 3299 1805

Highlights:

- tungsten alloys were investigated for their deuterium retention
- The "heavy alloy" HPM 1850 retains similar amounts as pure tungsten
- The self-passivating alloy W10Cr0.5Y retains 10 times more
- **Deuterium retention in tungsten based materials for fusion applications**
- 2 H. Maier<sup>a</sup>\*, T. Schwarz-Selinger<sup>a</sup>, R. Neu<sup>a,b</sup>, C. Garcia-Rosales<sup>c,d</sup>, M. Balden<sup>a</sup>, A. Calvo<sup>c,d</sup>, T.
- 3 Dürbeck<sup>a</sup>, A. Manhard<sup>a</sup>, N. Ordás<sup>c,d</sup>, T. F. Silva<sup>a,e</sup>
- <sup>a</sup> Max-Planck-Institut für Plasmaphysik, Boltzmannstrasse 2, 85748 Garching, Germany
- <sup>b</sup> Technische Universität München, Boltzmannstrasse 15, 85748 Garching, Germany
- 6 Ceit-IK4, Paseo de Manuel Lardizabal 15, 20018 San Sebastian, Spain
- <sup>d</sup>Universidad de Navarra, Tecnun, Paseo de Manuel Lardizabal 13, 20018 San Sebastian,
- Spain
- <sup>e</sup> Instituto de Física da Universidade de São Paulo, Rua do Matão, trav. R 187, 05508-090 São
- Paulo, Brazil
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#### **Abstract**

- The tungsten "heavy alloy" HPM 1850, a liquid-phase sintered composite material with two
- weight percent Ni and one weight percent Fe as well as the self-passivating tungsten alloy W-
- 10Cr-0.5Y, a high temperature oxidation resistant alloy with 10 weight percent of Cr and 0.5
- weight percent of Y were investigated with respect to their deuterium retention. The samples
- 17 were deuterium loaded in an electron cyclotron resonance plasma up to a fluence of  $10^{25}$ m<sup>-2</sup>.
- The deuterium retention was then investigated by Nuclear Reaction Analysis and by Thermal
- Desorption. In HPM 1850 the observed deuterium amount was similar to pure tungsten,
- however the outgassing behaviour during thermal desorption was considerably faster. In W-
- 21 10Cr-0.5Y the released deuterium amount during thermal desorption was about one order of
- magnitude higher; by comparison of nuclear reaction analysis and thermal desorption this was
- attributed to faster diffusion of deuterium into the bulk of the material.

 Keywords: deuterium retention, tungsten heavy alloy, tungsten self-passivating alloy, nuclear reaction analysis, thermal desorption

## **1 Introduction**

- The general present-day assumption for the choice of plasma-facing materials in future fusion
- devices is to use tungsten for both, the divertor and the main chamber wall, see for instance
- [1]. These applications, however, pose very different requirements to the materials properties.
- While a divertor target material must have sufficient toughness to withstand high thermo-
- mechanical loads, a main chamber plasma-facing material must protect the underlying steel
- structures from erosion without bearing high thermo-mechanical loads. The tritium retention
- in the near-surface region of the plasma-facing material, the re-emission, and the diffusion of
- tritium into the bulk are of importance for the tritium inventory in future fusion devices [2].
- This is especially important for main chamber materials because of the large surface area of
- the main chamber first wall in a fusion reactor.
- In the European fusion community a long-term materials development programme has been
- conducted in the frame of the EUROfusion Workpackage Materials, which also included the
- development of several tungsten-based materials for divertor as well as main chamber wall
- application [3, 4]. With respect to the plasma-material interaction properties the fusion
- community can readily refer to well-reviewed data for pure tungsten [5, 6]. There are,
- however, no data available for tungsten-based materials, which are new in the fusion
- community. The purpose of this contribution is to present first data for two tungsten-based
- materials with respect to their deuterium retention and to compare this with a reference
- tungsten material. For this purpose deuterium was implanted from an electron cyclotron
- resonance plasma and the retention was analysed by Nuclear Reaction Analysis (NRA) and
- Thermal Desorption (TD). This activity was performed in the frame of the EUROfusion
- Workpackage Plasma-Facing Components (PFC), see for instance [7].
- The materials under investigation here are a so-called tungsten heavy alloy, which could be
- <span id="page-4-0"></span>employed as an alternative divertor target material [8], and a self-passivating tungsten alloy
- <span id="page-4-1"></span>[9], an oxidation-resistant material, which was developed in the frame of the EUROfusion
- Workpackage Materials to be employed on the main chamber wall of a fusion reactor. These
- materials will be described in more detail in section 2. Section 3 gives the experimental
- details and procedures. The experimental results and a discussion are presented in section 4.

## **2 The Materials**

#### **2.1 Tungsten Heavy Alloy**

 The so-called tungsten heavy alloy is a two-phase composite material consisting of tungsten powder particles in a matrix of a metal with lower melting point. It is produced by liquid phase sintering. In the original publication either copper or nickel were proposed as matrix material [10]. Here we investigate a commercially available material of tungsten with two 64 weight percent nickel and one weight percent iron. The density of this material is  $18.5$  g/cm<sup>3</sup>; it was purchased from HC Starck Hermsdorf GmbH, Germany and is labelled HPM 1850.

- After a thorough pre-characterisation including high heat flux testing, which was reported in
- [11], this material was employed as a divertor target material in the tokamak ASDEX
- Upgrade. Because of its better mechanical properties the use of HPM 1850 was proposed to
- eliminate problems with cracking of bulk tungsten tiles, which were reported in [12]. The
- positive results of this measure are reported in detail in [\[8\]](#page-4-0).
- Figure 1 shows the microstructure of HPM 1850: It consists of tungsten powder grains of sizes of a few 10 µm up to more than 100 µm surrounded by the nickel/iron matrix.
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#### **2.2 Self-Passivating Tungsten Alloy**

- <span id="page-5-0"></span> In the case of a loss of coolant the plasma-facing surfaces of a fusion reactor can reach high temperatures due to residual decay heat. In the European Power Plant Conceptual Studies PPCS it was shown that the outboard first wall of the reactor concepts A, B, and C can reach temperatures in excess of 1000°C within days or tens of days [13]. In such a situation the 81 ingress of oxygen would lead to the formation of volatile and radioactive  $WO_3$ , see [14] and 82 references therein. For this reason it was proposed in  $[14]$  to use self-passivating alloys as plasma-facing materials on the first wall. In such an alloy the alloying elements form a stable oxide layer on the surface when exposed to oxygen which prevents further oxidation of the material. The composition we investigate in this contribution consists of tungsten with 10 weight percent of chromium and 0.5 weight percent of yttrium as alloying elements, labelled W-10Cr-0.5Y. It is produced at CEIT, San Sebastian by mechanical alloying and hot isostatic pressing. Its density is 99.7% of the theoretical value from the rule of mixture. An SEM image
- of this material is shown in figure 2. The material consists of a tungsten-rich phase, which
- appears in light grey and a chromium-rich phase in black. The typical grain size is on the
- 91 order of 100 nm. Further details can be found in [\[9\]](#page-4-1).
- 

#### **2.3 Tungsten Reference Material**

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- The tungsten reference material was exposed to the deuterium plasma in parallel with the respective self-passivating alloy and heavy alloy samples for comparison.
- It is a hot-rolled polycrystalline tungsten material from Plansee SE with a specified purity of
- 99.7 %. The typical grain size is in the micrometre range, as shown in figure 3. A detailed
- microstructural analysis of this material including the effect of an annealing step at 930°C can
- be found in [15].

#### **3 Experimental Procedures**

 Samples of W-10Cr-0.5Y, HPM 1850, and tungsten reference material were polished and pre- characterised by microscopy. After outgassing them in vacuum at 930°C the samples were loaded with deuterium in an ECR plasma device. Then the deuterium retention was analysed by Nuclear Reaction Analysis (NRA) and subsequently Thermal Desorption (TD) was performed. In the following the procedures for deuterium loading as well as NRA and TD analysis are given in more detail.

#### **3.1 Deuterium Loading**

- 
- Deuterium loading of the samples was performed by exposing them to a deuterium plasma
- 112 from an ECR plasma source described in [16]. The plasma ions from this source are mostly
- $D_3^+$  ions. Only 3 % of the impinging D flux arrives at the samples in the form of  $D^+$  and  $D_2^+$  ions.
- D loading was performed at a sample temperature of 100°C and a bias voltage corresponding
- 116 to 38 eV/D for the  $D_3^+$  ions. Three sets of samples were exposed to D fluences of  $10^{23}$ m<sup>-2</sup>,
- 117  $10^{24}$ m<sup>-2</sup>, and  $10^{25}$ m<sup>-2</sup>, respectively. At a flux of  $10^{20}$ m<sup>-2</sup>s<sup>-1</sup> this corresponded to exposure times
- of 0.3h, 3h, and 30h. With each set one sample of tungsten reference material was also
- exposed.
- **3.2 Nuclear Reaction Analysis**
- 

 Nuclear Reaction Analysis was performed using the 3 MeV tandem accelerator at the Max-Planck-Institut für Plasmaphysik in Garching.

- 124 Using the  $D(^{3}He,p)\alpha$  reaction at <sup>3</sup>He beam energies up to 6.0 MeV and detecting the high
- energy protons allows probing the D content in the W samples up to a depth of approximately
- 126  $\,$  10  $\mu$ m. Using the software NRADC to deconvolute the proton spectra from different  $\rm{^{3}He}$
- beam energies, concentration depth profiles can be constructed. In our analysis we used 6
- different energies between 0.69 MeV and 6.0 MeV. NRADC is described in detail in
- reference [17]. It is based on a Markov chain Monte Carlo approach and is especially
- designed and optimised for the construction of trace element depth profiles. The software uses
- the SIMNRA program [18] to compute proton spectra. A subset of our data was also analysed
- using the independent computer code MULTISIMNRA [19]. MULTISIMNRA is a general
- purpose tool for analysing sets of ion beam spectra by running multiple instances of SIMNRA
- and performing multi-dimensional fits. The deuterium concentration depth profiles produced
- by using NRADC and MULTISIMNRA coincide within the error ranges given by the
- respective software package.
- **3.3 Thermal Desorption**
- 
- Thermal desorption was performed in the setup described in reference [20]. Basically it
- consists of a stainless steel vacuum vessel with a quadrupole mass spectrometer at a base
- 141 pressure in the  $10^{-10}$  mbar range. Via a metal glass transition a quartz glass tube is attached to
- 142 the vessel, which contains the samples at a base pressure in the  $10^{-8}$  mbar range. The samples
- can be heated up to 1050°C by moving a tubular furnace over the quartz glass tube. In our
- experiments a heating ramp of 15 K/min was used.
- For the pure tungsten reference samples the temperature was ramped up to 1050°C. For the
- alloys the heating ramps were stopped at 850°C. This was done because of the higher vapour
- pressures of the alloying metals as compared to that of pure tungsten.
- 148 The mass-4 signal of the mass spectrometer when detecting  $D_2$  gas is calibrated by injecting a
- calibrated flux of deuterium gas into the vessel. For deuterium arriving at the mass
- spectrometer in the form of HD molecules a calibration using a calibrated volume and a
- spinning rotor pressure gauge was used as explained in reference [21]. A part of the released
- 152 deuterium, however, arrives at the mass spectrometer in the form of water HDO or  $D_2O$ ,
- respectively, i.e. at the masses 19 and 20. For these a direct calibration factor is not available.
- In our measurements the relative amount of deuterium released in the form of water decreases
- substantially with increasing implantation fluence. Assuming the same sensitivity of the setup
- 156 for HDO and  $D_2O$  as for the HD signal from mass 3, we estimate it to range from about 20-
- 35% at the lowest applied implantation fluence to 4-7% at the highest fluence.

#### **4 Results and Discussion**

#### **4.1 HPM 1850**

 Figure 4 shows the integrated amounts of deuterium released during the thermal desorption ramp as a function of implantation fluence for two HPM 1850 samples at each fluence in comparison with the reference tungsten material. In these datasets the mass spectrometer 165 signals for all masses discussed in section 3c are included, i.e.  $D_2$ , HD, HDO, and  $D_2O$ . As the figure shows, the total released amount is the same for HPM 1850 and the reference tungsten within a factor of 2. The fact that the trend is reversed from the lowest to the highest implantation fluence can possibly be related to the uncertain contribution of the masses 19 and 20, as discussed in section 3c.

Figure 5 shows thermal desorption data from two HPM 1850 samples implanted with a

171 deuterium fluence of  $10^{25}$ m<sup>-2</sup> in comparison with a reference tungsten sample implanted with 172 the same fluence. The temperature ramp is 15 K/min. For the pure tungsten sample the

temperature was ramped up to 1050°C while for the alloy it was limited to 850°C. As the

figure shows, for both HPM 1850 samples the deuterium release occurred at lower

temperatures than for the tungsten reference sample. This indicates either a faster out-

diffusion or lower trap binding energies in the HPM 1850 material as compared to the

reference tungsten material.

In reference [22] hydrogen diffusion coefficients for the pure metals nickel and iron are given.

According to this reference the activation energy for hydrogen diffusion in nickel is very

similar to the tungsten case, i. e. about 0.4 eV, while in iron the activation energy is

- significantly lower. Reference [23] shows in detail that the activation energy for hydrogen
- diffusion in Ni/Fe alloys actually is a function of the alloy composition and depends on the
- thermal history of the sample. Depending on the heat treatment, which was also performed in

our case, the activation energy for hydrogen diffusion in Ni/Fe alloys can have a minimum of

- less than 0.2 eV in the compositional range of 70% to 80% Ni. As this is close to the
- composition of the matrix material in HPM 1850, this could be the reason why the out-
- diffusion occurs considerably faster during the temperature ramp in the desorption
- experiment. However, this would only hold for hydrogen atoms diffusing in the Ni/Fe matrix.
- Reference [24] gives enthalpies of solution for hydrogen in pure nickel or pure iron which are
- considerably lower than that of tungsten. Data on the enthalpy of solution of deuterium in a
- Ni/Fe alloy with the actual composition of our sample material are not available. However, if
- we assume the trend given for the pure materials to hold also for the alloy, then this would
- additionally support the faster out-diffusion process, because hydrogen atoms would
- energetically prefer the Ni/Fe matrix from the tungsten grains. In summary this would mean
- that hydrogen atoms entering the matrix material would stay in there and leave the sample in a
- faster diffusion process compared to tungsten.
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#### **4.2 W-10Cr-0.5Y**

 Figure 6 shows the total released amounts of deuterium from the W-10Cr-0.5Y samples in comparison with the reference tungsten material. Here we have 2 data points for the highest 203 and lowest deuterium implantation fluence at  $10^{23}$ m<sup>-2</sup> and  $10^{25}$ m<sup>-2</sup>, and only one data point at 204 the intermediate fluence of  $10^{24}$ m<sup>-2</sup>. The data for the tungsten reference material are the same as in figure 4 and are plotted here for comparison.

 The figure clearly shows that the amount of deuterium released by the W-10Cr-0.5Y alloy is roughly an order of magnitude higher than the corresponding amount from the tungsten reference material. This observation holds for the whole investigated implantation fluence range, which spans 2 orders of magnitude, and shows to be well reproducible in the cases where two samples per fluence were employed.

211 In figure 7 a comparison of the TD data from figure 6 with the corresponding NRA data from the same samples is shown: As in figure 6 the TD data from the W-10Cr-0.5Y samples are represented by the blue crosses and green squares and the TD data from the tungsten reference samples are represented by closed black squares. For all samples the NRA data were obtained several days before outgassing the samples in the TD setup. They are shown as blue and green open circles and as black open squares, respectively, for the two sample types. The comparison shows that with increasing implantation fluence the integrals of the TD data increase systematically, while the integrated NRA data stay approximately constant for the 219 two higher fluences of  $10^{24}$ m<sup>-2</sup> and  $10^{25}$ m<sup>-2</sup>. At the highest fluence there is a discrepancy of about one order of magnitude. In contrast to this, for the tungsten reference material the TD

- data and the NRA data for the two higher fluences coincide approximately.
- In our interpretation this comparison shows that with increasing fluence, which corresponds to
- increasing implantation time, in the case of W-10Cr0.5-Y the implanted deuterium diffuses
- deeper into the material, such that at the two longer implantation times in our experiment most of the deuterium already diffused beyond the information depth of our NRA depth
- profiling measurements, which is about 10 µm. Therefore the TD analysis shows a larger
- amount of deuterium, which cannot be detected by the NRA measurements, since it is mostly
- trapped beyond the NRA information depth.
- A similar observation was made in a different tungsten alloy: In reference [25] it was
- <span id="page-8-0"></span>observed that alloying of tungsten with 1% or 5% of tantalum, respectively, led to an increase
- of the retained amount of deuterium by roughly one order of magnitude for implantation
- 232 fluences in the range of  $10^{24}$ m<sup>-2</sup>, which is similar to the fluence range investigated here. In the
- 233 data shown in [\[25\]](#page-8-0), however, there is a clear fluence dependent increase of the relative
- deuterium retention with increasing implantation fluence, which does not show in our data.
- 235 Different from our interpretation presented here, the authors of  $[25]$  conclude from a
- comparison of their results with modelling, that the increased retention is due to an increase of
- the trap density in the material caused by tantalum alloying rather than due to deeper
- diffusion.

#### **5 Summary, Conclusions, and Outlook**

We performed an investigation of the deuterium retention in two materials, which are new in

242 the fusion community: the tungsten heavy alloy HPM 1850, which is a liquid phase sintered

material of tungsten with 3 weight percent of nickel and iron and a self-passivating alloy of

tungsten with 10 weight percent of chromium and 0.5 weight percent of yttrium, W-10Cr-

- 0.5Y, which is a material resistant to high temperature oxidation.
- 246 Samples were loaded with deuterium from an ECR plasma with fluences of  $10^{23}$ m<sup>-2</sup>,  $10^{24}$ m<sup>-2</sup>,
- 247 and  $10^{25}$ m<sup>-2</sup>, respectively. These samples were then analysed with Nuclear Reaction Analysis 248 with a  ${}^{3}$ He beam of energies in between 0.69 MeV and 6.0 MeV, which allows deuterium
- depth profiling up to a depth of about 10 µm. Subsequently the samples were outgassed in a
- Thermal Desorption setup. These results were compared to results from a pure tungsten
- reference material.

It was found that the amount of deuterium released from HPM 1850 samples is very similar to

that from pure tungsten, whereas the outgassing occurs faster during the thermal desorption

temperature ramp. This was interpreted in terms of faster out-diffusion of deuterium through

- the Ni/Fe matrix transport channel.
- For the W-10Cr-0.5Y material it was found that the amount of deuterium released during the
- thermal desorption temperature ramp is about one order of magnitude higher than the

corresponding amount from our reference tungsten material. This is valid for all investigated

implantation fluences. By comparing this result with the corresponding result from Nuclear

Reaction Analysis depth profiling it was concluded that this observation is due to faster

- diffusion of deuterium beyond the information depth of the ion beam analysis into the bulk of
- the samples.
- For both of the sample materials investigated in this contribution our interpretation of the
- different behaviour when compared to our reference tungsten material involves diffusion.
- Therefore the next step will be the investigation of the temperature dependence of the
- deuterium retention. This will shed more light onto the question whether the diffusion
- coefficients in the W-10Cr-0.5Y alloy in one case and in the Ni/Fe matrix of HPM 1850 in the
- other are responsible for the observed differences.
- The final step of this work will be the investigation of deuterium retention in samples
- irradiated with high energy ions. This allows the simulation of neutron induced radiation
- damage by using an MeV accelerator. In the process of the European fusion roadmap the
- European materials assessment group called the lack of data on the effect of irradiation on the
- tritium retention of tungsten materials a "highest impact project level risk" for DEMO [26].
- **Acknowledgement**

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#### **Paper length**

Main text: 3632 words Figure captions: 395 words Figures total: 130 + 130+130+ 150+176 +150 + 150 = 1016 words Total: 3632 + 395 + 1016 = 5043 words. (limit is 5500 words)

Figure captions:

Figure 1: SEM image of HPM 1850 in backscatter electron contrast. The different grey shades are due to grain orientation contrast; the dark area surrounding the tungsten grains is the nickel/iron matrix. The scale bar at the bottom right is 200  $\mu$ m.

Figure 2: SEM image of W10Cr0.5Y in backscatter electron contrast. The scale bar at the bottom right is 5 µm. The dark areas are a chromium-rich phase, see text.

Figure 3: SEM image of the tungsten reference material in backscatter electron contrast. The different grey shades are due to grain orientation contrast. The scale bar at the bottom right is 5  $\mu$ m. The image was taken after the experiments

Figure 4: Total released D amounts in the thermal desorption experiment as a function of implantation fluence for HPM 1850 and tungsten reference material. For HPM 1850 there are two data points per fluence: Sample #1 blue crosses, sample #2 red circles. The black squares are the data from the tungsten reference material.

Figure 5: Thermal desorption data for two HPM 1850 samples (solid line/ blue and dashed line/red) and one tungsten reference sample (black solid line with symbols) implanted with the highest deuterium fluence of  $10^{25}$  m<sup>-2</sup>. The left ordinate axis gives the mass spectrometer mass-4 signal in counts per second. The right ordinate axis shows the furnace temperature. The temperature ramp is shown as a straight line up to 1050°C for the reference tungsten sample. The dashed line indicates the end of the ramp at 850°C for the HPM 1850 samples. The temperature ramp started from room temperature.

Figure 6: Total released D amounts in the thermal desorption experiment as a function of implantation fluence for W10Cr0.5Y and tungsten reference material. Blue crosses and green squares: W10Cr0.5Y; black squares: tungsten reference. For W10Cr0.5Y there are two data points for the highest and lowest implantation fluence and one data point for the intermediate fluence. The tungsten reference data are the same as in figure 3.

Figure 7: Comparison of the total released D in the thermal desorption (TD) measurements with the integrated depth profile from Nuclear Reaction Analysis (NRA). TD data: blue crosses and green squares for W110Cr0.5Y and black squares for the tungsten reference material. NRA data: blue and green circles for W10Cr0.5Y and black open squares for the tungsten reference material. The TD data are the same as in figure 6.

Figure 1, 7.5 \* 6.5 cm



 $6.5$  cm  $*$  20 words/cm = 130 words

Figure 2 7.5 \* 6.5 cm



 $6.5$  cm  $*$  20 words/cm = 130 words

Figure 3: 7.5\*6.5 cm



Figure 4: 7.5\*7.5 cm



7.5 cm \* 20 words/cm = 150 words

Figure 5: 7.5\*8.8 cm



8.8 cm \* 20 words/cm = 176 words

Figure 6: 7.5\*7.5 cm



7.5cm \*20 words/cm = 150 words

Figure 7: 7.5\*7.5 cm



7.5 cm \*20 words/cm = 150 words