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# Recent progress on improving ICRF coupling and reducing RF-specific impurities in ASDEX Upgrade

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**Abstract.** The recent scientific research on ASDEX Upgrade (AUG) has greatly advanced solutions to two issues of Radio Frequency (RF) heating in the Ion Cyclotron Range of Frequencies (ICRF): (a) the coupling of ICRF power to the plasma is significantly improved by density tailoring with local gas puffing; (b) the release of RF-specific impurities is significantly reduced by minimizing the RF near field with 3-strap antennas. This paper summarizes the applied methods and reviews the associated achievements.

## 1 Introduction

The coupling of Radio Frequency (RF) power in the Ion Cyclotron Range of Frequencies (ICRF) to the plasma depends sensitively on the Scrape-Off Layer (SOL) density because the fast wave is evanescent below the cut-off density (typically in the order of  $10^{18} \text{ m}^{-3}$  in nowadays tokamaks). The ICRF power coupling can be greatly enhanced by increasing the density in front of the antennas with local gas puffing. In ASDEX Upgrade (AUG) tokamak, it is found that compared to divertor gas puffing, midplane gas puffing close to the antennas increases the ICRF power coupling most significantly and top gas puffing increases it in a smaller level. The 3D edge plasma fluid and neutral transport code EMC3-EIRENE [1] is used to model the SOL density during different gas puffing scenarios in AUG. Antenna codes such as FELICE [2], TOPICA [3] and RPLICASOL [4] are then used to calculate the antenna coupling resistance. Good agreement is found between the simulation and experimental results [5, 6].

The RF enhanced sheath potential is due to the image current driven on the antenna box. New 3-strap antennas with a novel approach to reduce those undesired currents through adjusting properly the phase and amplitude of the current in the straps have been installed in ASDEX

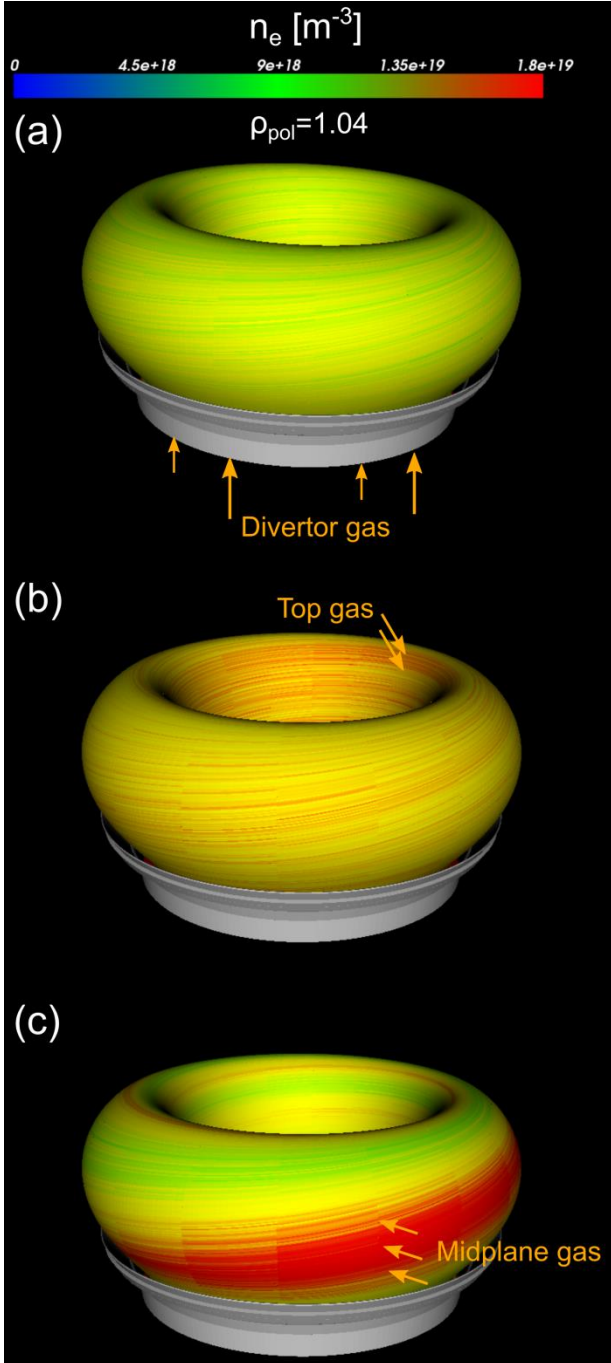
Upgrade. The recent results have shown that compared to 2-strap antennas, the use of 3-strap antennas reduces the RF-specific tungsten release significantly [7]. The SOL density convection due to the inhomogeneous RF-sheath potential was measured with poloidally distributed reflectometers embedded in one 3-strap antenna [8]. The measured density convection can clearly be reproduced in the self-consistent simulations by running the EMC3-EIRENE, RPLICASOL and SSWICH [9] codes in an iterative way [10]. Progress of these studies will be reviewed in the paper.

## 2 Improvement of ICRF coupling

The basic strategy of improving the ICRF power coupling is to decrease the width of the evanescence layer between the antenna strap and the fast wave cut-off density position by increasing the local density in the SOL with main chamber deuterium (D2) gas puffing (or so-called local gas puffing). Both experiments and simulations in AUG [5, 6, 11-13] showed that significant increase of the effective coupling resistances was achieved by moving the gas source from the divertor to the main chamber. The core plasma properties were kept close to identical during different gas puffing.

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**Fig. 1.** 3D electron density in AUG during (a) divertor gas puffing; (b) top gas puffing; (c) midplane gas puffing. The densities are plotted on the same flux surface. The same gas puff rate is used ( $1.2 \times 10^{22}$  el/s) and the gas puffing locations and directions are indicated by the arrows.

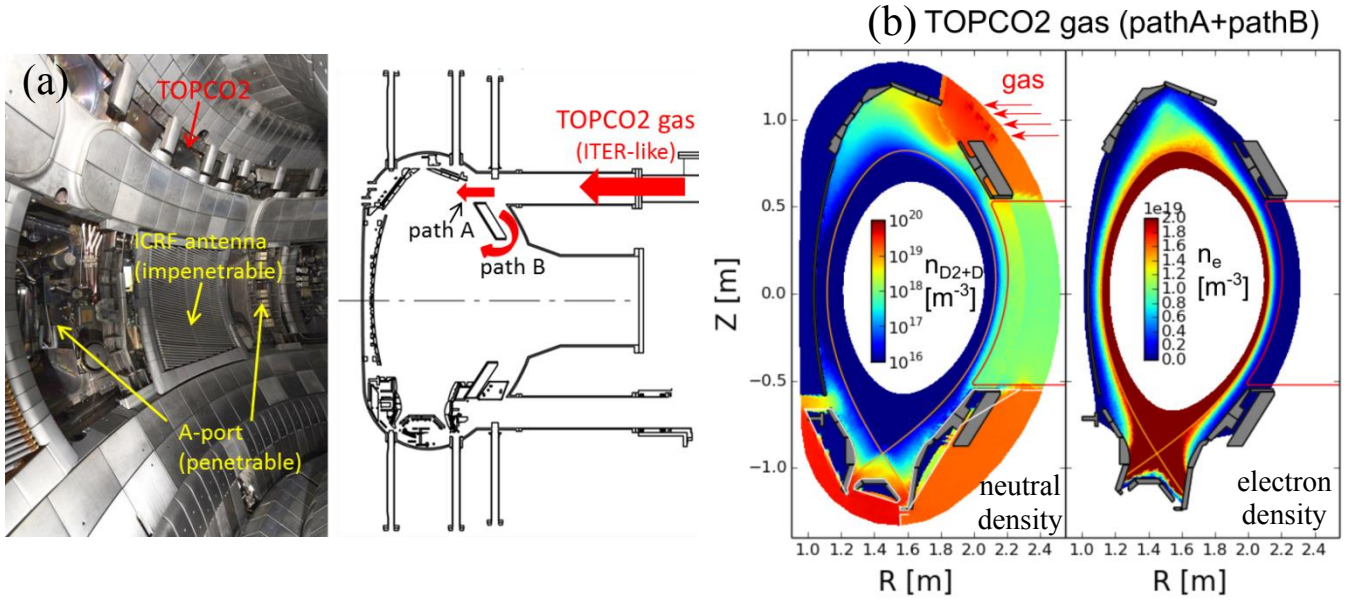
Here, the EMC3-EIRENE code is used to calculate the SOL plasma density in the presence of different gas sources. A toroidal  $360^\circ$  grid with a high resolution in the SOL is used. All the plasma facing components as well as the realistic gas source positions are taken into account. Moreover, the SOL plasma parameters, mainly the upstream density and temperature profiles and the downstream power and particle fluxes are fitted to the experimental ones by choosing appropriate particle and energy transport coefficients [5, 6]. SOL boundary conditions such as the separatrix density and power

across the separatrix are set as the same as those in the experiments. No impurity is considered in the simulations and the part of power radiated by impurities is subtracted from the total heating power.

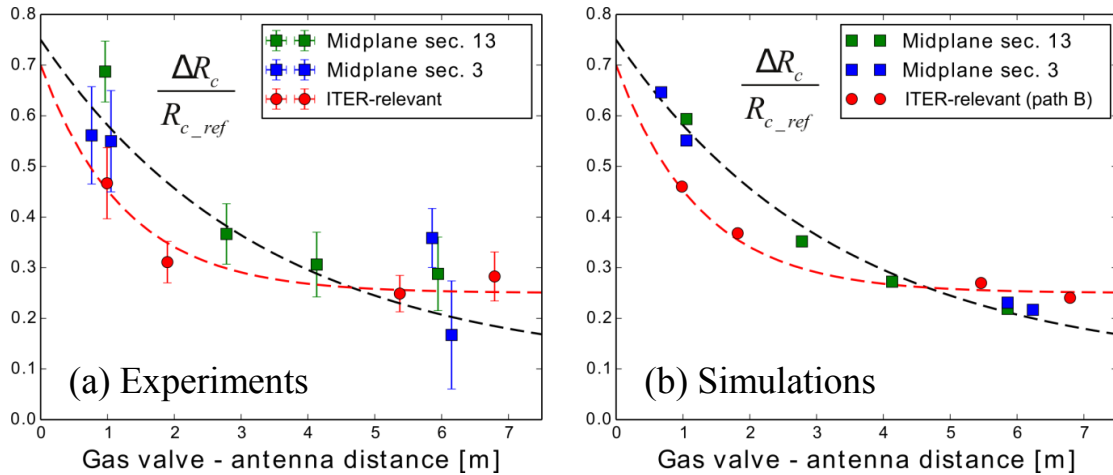
An example of the simulated SOL electron densities during divertor, top and midplane gas puffing is shown in Fig. 1. Compared to divertor gas puffing (reference case), top gas puffing increases the SOL density almost uniformly toroidally, but to a small extent; midplane gas puffing increases the SOL density very significantly and locally. The density profiles measured by the reflectometry at particular toroidal positions are in good agreement with those in simulations [5, 6]. The increase of local density leads to a shift of the fast wave cut-off density layer towards the vessel. This shift is  $\sim 0.7$ cm during top gas puffing and is  $\sim 2.0$ cm in locations near the gas valve during midplane gas puffing. The shift of the fast wave cut-off density layer decreases the width of evanescence layer and increases the ICRF power coupling.

The mechanisms of the local density increase by local gas puffing are summarized as follows. Firstly, a localized neutral gas cloud is developed in front of the top or midplane gas valve. Due to ionization ( $T_e > 2$ eV), a localized high plasma density cloud develops in the SOL. Charged particles in this high density cloud transport very quickly along the magnetic field lines with a consequent increase of the density in the regions magnetically connected with the valves. Field lines starting from the top density cloud towards the midplane spread widely and lead to an evenly increased SOL density. Field lines starting from the midplane density cloud are much more concentrated and result in a localized density increase. Additionally, the plasma temperature at the locations of the density cloud are lowered, because the outward plasma flows (particle convection) and ionizations provide energy sinks. The plasma pressure (density $\times$ temperature) along the same flux tubes are the same, thus density gradients are developed along the same flux tubes. The largest density is expected to be at location where the ionization is largest.

Based on the 3D density calculated with EMC3-EIRENE, antenna codes such as FELICE (1D) [2] or RPLICASOL (3D) [4] are then used to calculate the effective antenna coupling resistance. For FELICE calculations, the 3D density has to be averaged into a 1D radial profile, and this 1D averaging can bring additional errors to the final results. Nevertheless the calculated resistances with FELICE are in qualitative agreement with the experimental ones [5]. They indicate that compared to divertor gas puffing in H-mode plasmas, top gas puffing increases the coupling resistance by  $\sim 25\%$  for all the antennas; midplane gas puffing increases this value by  $\sim 120\%$  for the antenna close to the gas valve and this increase decays exponentially (decay length  $\sim 3.7$ m) as the distance between the antenna and the gas valve becomes larger.



**Fig. 2.** (a) Outer top gas puffing (TOPCO2, ITER relevant) and the surrounding wall structures in AUG; (b) neutral and electron density during outer top gas puffing. Figure reproduced with permission from [6].



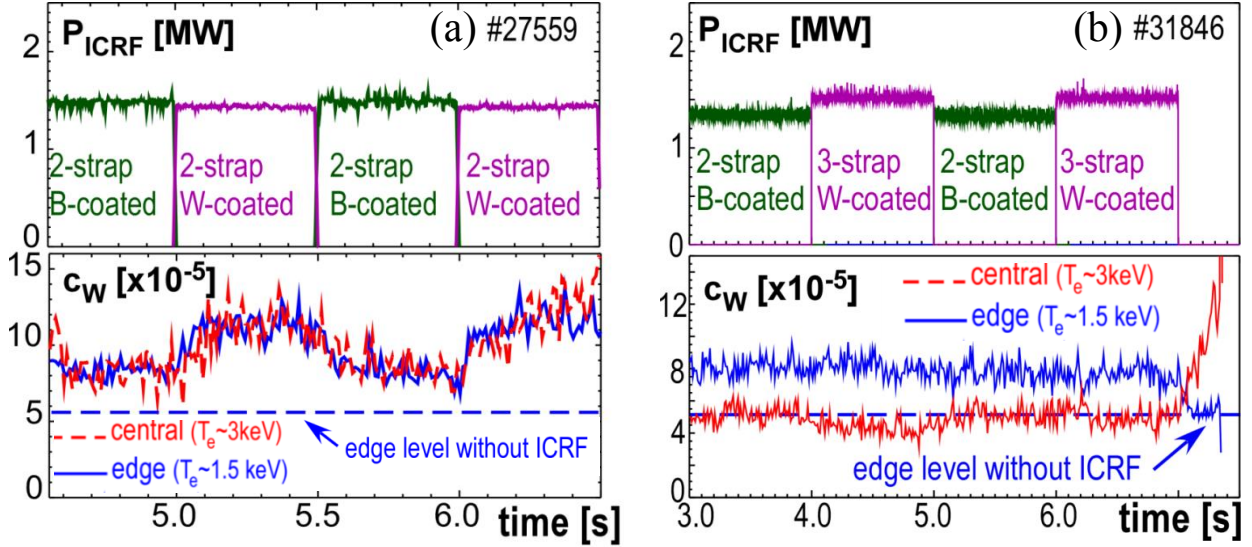
**Fig. 3.** Relative increase of coupling resistance  $\Delta R_c/R_{c\_ref}$  obtained from (a) experiments; (b) simulations with the EMC3-EIRENE and RAPLICASOL codes. Figure reproduced with permission from [6].

The previously discussed top gas sources are located at the inner top of the vessel. In addition, we have studied the outer top gas puffing (TOPCO2) in AUG L-mode plasmas [6]. This gas puffing is more ITER relevant, because in ITER the fueling gas is planned to be puffed from the outer top positions. The distribution of this gas depends on the surrounding wall structures and is different in different machines. In AUG, the outer top gas enters the plasma either through the top windows (path A) or through the A-ports in the midplane (path B), Fig. 2(a). It is shown that a localized neutral cloud is developed in the top while a more homogenous one is formed in the midplane, Fig. 2(b).

The calculated relative increase of coupling resistances with the RAPLICASOL code is in quantitative agreement with the experimental values shown in Fig 3. The largest increase of coupling resistance is  $\sim 70\%$  in L-mode plasmas with midplane gas puffing (gas puff rate  $\sim 6.5 \times 10^{21}$  el/s). As for the outer top gas puffing

(ITER relevant), the experimental results are in better agreement with simulations when only path B is assumed to be open (i.e. with the part of gas clouds only in the midplane). It is found that the antenna closest to the gas valve has the largest coupling resistance increase and this increase decays with the gas valve - antenna distance. Considering the fact that about half of the injected gas passes through path A and the corresponding gas cloud in the top is magnetically connected to antenna 3 (with a gas valve - antenna distance  $\sim 5.4$ m), the effect of the localized high neutral density on the local increase of plasma density plays a more important role than the effect of field line connections. Indeed, in the simulations the top gas cloud associated with path A increases the SOL density in a much more toroidal uniform way [6].

From our previous studies, it can be concluded that the poloidal distributed gas valves close to the ICRF antennas are the best option in terms of maximizing the



**Fig. 4.** Central and edge W concentrations during ICRF heating with (a) B-coated and W-coated 2-strap antennas; (b) B-coated 2-strap and W-coated 3-strap antennas. Figure reproduced with permission from [7, 15].

ICRF power coupling. This conclusion is expected to be also valid for future machines such as ITER and DEMO.

An additional benefit of local gas puffing is the reduction of total outer wall sputtering and core W concentration [14]. This is attributed to the local decrease of plasma temperature in the far SOL and the decrease of sheath potential in front of the antennas during local gas puffing. More investigations are underway to understand the mechanisms. Furthermore, the influence of magnetic perturbation fields and MHD modes on the SOL density and thus on the ICRF coupling properties is now being studied [21]. Possible synergies with gas puffing will also be addressed in the future.

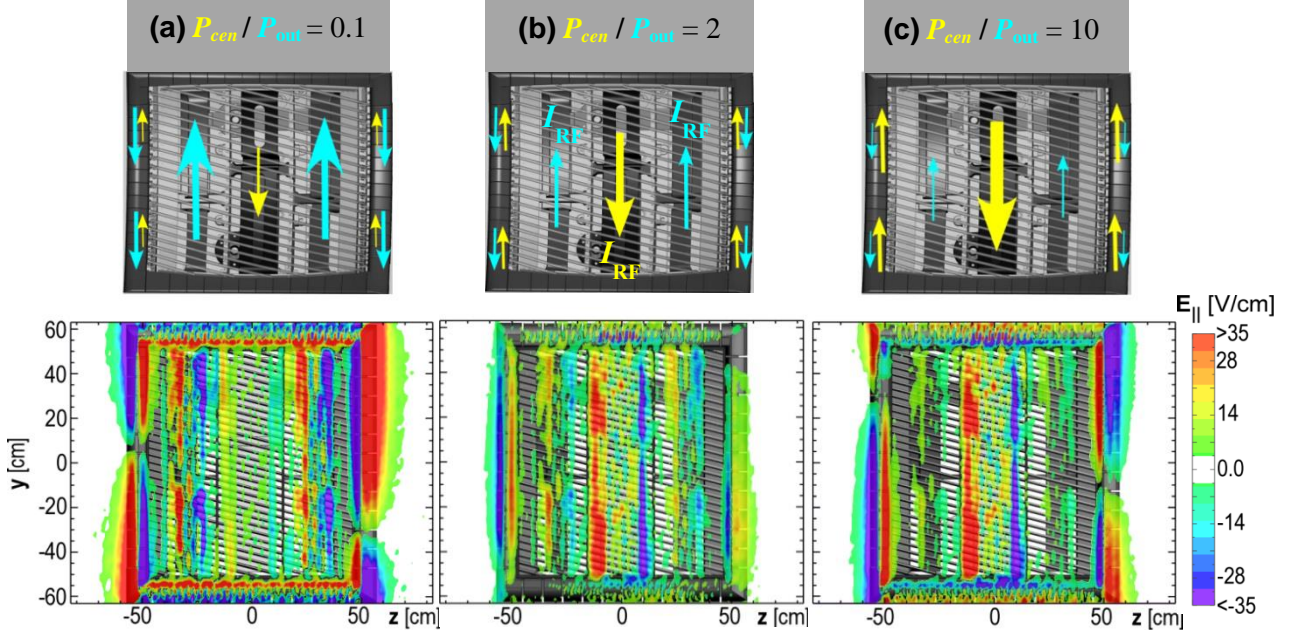
### 3 Reduce RF-specific impurities

ICRF antennas excite fast wave via oscillating currents on the radiating straps. However the oscillating currents induce image currents on the antenna box, in particular on the antenna limiters. These image currents can generate  $E_{\parallel}$  fields (parallel to the equilibrium magnetic field) in the vicinity of the antennas. Subsequently an increase of sheath potential is driven by the  $E_{\parallel}$  fields. The large potential drop across the sheath then accelerates ions towards the wall, leading to considerable physical sputtering and even hot spots if the impacting ions are energetic enough. The hot spots can damage the wall components and the sputtering induced impurities can affect the plasma performances. In AUG (full tungsten (W) wall), substantial impurity radiation losses can be generated if the W concentration ( $c_W$ ) in the plasma core exceeds  $5 \times 10^{-5}$  [15].

The recent research activities in AUG have made significant progresses in reducing the ICRF-specific impurities. The first strategy is to coat the antenna limiters with a low-Z material boron (B) instead of W

since experiments suggest that the major ICRF-specific W sources are from the antenna limiters [16]. In 2012, limiters of two antennas were coated with boron. The experimental results (Fig. 4(a)) clearly show that a reduction of W concentration by a factor of two both in the edge and central plasma has been achieved during ICRF heating with B-coated 2-strap antennas. It is indicated that the antenna limiters account for more than half of the increase in W concentration during ICRF heating. In these experiments, a standard H minority scheme with dipole phasing was used. The shifting of sputtering from tungsten to boron is less problematic since boron content in core plasma is more tolerable and causes less power radiation. The B-coated antennas allow the ICRF heating to operate in a wider range of plasma conditions. The B-coating is only a remedy of the high-Z impurity production induced by ICRF and not a solution of the problem associate with the RF fields close to the antenna.

The long-term strategy to reduce the ICRF-specific impurities involves optimizing the antenna design so that the generated  $E_{\parallel}$  fields can be minimized. One of the strategies was to use broad-limiter antennas to reduce the  $E_{\parallel}$  fields near the antenna limiters. A reduction of W release of up to  $\sim 40\%$  in a single antenna operation was found in experiments operated with broad-limiter antennas [15]. The other more advanced approach is the design and use of 3-strap antennas in AUG. The basic idea of this approach is to minimize the RF image currents on the antenna limiters by balancing the dipole contributions of the central strap with that of the outer straps [7]. In 2015, a pair of tungsten (W) coated 3-strap antennas has been installed in ASDEX Upgrade. The experimental results (Fig. 4(b)) show that the W-coated 3-strap antennas lead to almost the same values of edge and core W concentration as those for the B-coated 2-strap antennas. The heating effectiveness of the W-coated 3-strap antenna is not lower than that of



**Fig. 5.** Image currents and parallel electric fields induced by 3-strap antenna with power balance of (a)  $P_{cen}/P_{out} = 0.1$ ; (b)  $P_{cen}/P_{out} = 2$ ; (c)  $P_{cen}/P_{out} = 10$ . In these cases dipole phasing is used. Figure reproduced with permission from [18].

B-coated 2-strap [7] while the ICRF heating power, plasma current, total input and radiated power are almost the same in the studied discharge. Together with the previous results (Fig. 4. (a)), it can be pointed out that the use of W-coated 3-strap antennas reduces the W concentration also by at least a factor of two compared to the W-coated 2-strap antennas.

A prominent rise of core W concentration after switching the ICRF power off (Fig. 4 (b)) is due to changes in the neoclassical transport of W. During ICRF heating, the core plasma density profiles are flattened and the core plasma temperature profiles are peaked. The flattening of density leads to reduced density gradients and thus reduced inward W transport while the peaking of temperature leads to increased temperature gradients and thus increased outward W transport [17]. As a result the core W concentration during ICRF heating is much lower than that without.

The final W concentration and W sputtering yield depend on the 3-strap antenna feeding configurations, including the phasing and power ratio between the central and outer straps ( $P_{cen}/P_{out}$ ,  $P_{cen}$  is the power on the central strap and  $P_{out}$  is the total power on the two outer straps, respectively). The lowest W concentration and the lowest W sputtering yield on the antenna limiters were observed when dipole phasing and  $P_{cen}/P_{out}$  in the range (1.5, 2.0) were used [7]. Non-optimized feeding configurations will cause high image currents and W sputtering yield. Such an example is shown in Fig. 5, in which large unbalanced image currents and  $E_{||}$  fields are generated on the antenna limiters when  $P_{cen}/P_{out}$  is too small (case (a)) or too large (case (c)). Even though, it is indicated from the numerical studies that the  $E_{||}$  fields and sheath potential induced by 2-strap antenna with typical feeding configurations (dipole phasing,

$P_{cen}/P_{out} = 1$ ) are still larger than those induced by 3-strap with non-optimized settings (dipole phasing,  $P_{cen}/P_{out} \sim 9$ ) [10].

Further experiments show that the local measurements of RF voltages at the side limiters of one 3-strap antenna correlate with the measured W sputtering yield. The calculated local  $E_{||}$  fields by TOPICA [3] at the same locations are qualitatively consistent with the measured RF voltages [18]. These findings provide evidences for links between the  $E_{||}$  fields, RF voltages and W sputtering yield.

The number of sputtered impurities not only depend on the sputtering yield but also on the number of impacting particles, i.e. on the local plasma density in the far SOL.  $E \times B$  drifts driven by the inhomogeneous sheath potential will however influence this local plasma density. Recently, progress has been made on understanding the ICRF induced density convection in the SOL from two numerical approaches [10, 19]: (a) EMC3-EIRENE simulations based on the measured potential; (b) Self-consistent simulations with several codes including EMC3-EIRENE, RAPLICASOL and SSWICH. In the first approach, the  $E \times B$  drifts are calculated from the measured potentials and are then used as prescribed drifts in EMC3-EIRENE. In the second approach, the output of EMC3-EIRENE (plasma density) is used as inputs in RAPLICASOL and SSWICH, the output of RAPLICASOL (the  $E_{||}$  fields) are used as inputs in SSWICH and the output of SSWICH (sheath potential) is used to calculate the  $E \times B$  drifts, which are then used as inputs in the EMC3-EIRENE code. The obtained results from these simulations are in agreement [19] or consistent [10] with the corresponding measurements.

## 4 Conclusions

This paper summarizes the recent progress of ICRF heating in ASDEX Upgrade tokamak on two aspects: improving the ICRF power coupling and reducing the ICRF-specific impurities.

The ICRF power coupling is improved by increasing the local density in front of the antennas with local deuterium gas puffing. Compared to divertor gas puffing, midplane gas puffing close to the ICRF antenna increases the ICRF power coupling most significantly (~120%) while top gas puffing increases it in a much smaller level (~25%) [5]. The effects of local gas puffing on local plasma density are attributed to the local high neutral density, magnetic field line connections and the local decrease of temperature [5, 20]. It can be extrapolated that for future machines, such as ITER and DEMO, local gas valves close to the antennas is the best option in terms of maximizing ICRF power coupling.

The tungsten concentration in the central and edge plasma was reduced by using one of the three antenna modifications: (1) the broad-limiter antenna (up to ~40% for single antenna operation) [15]; (2) the boron-coated antennas (by about a factor of 2) [15]; (3) the 3-strap antennas (by more than a factor of 2) [7]. The tungsten sources, especially those from the antenna limiters are reduced either by replacing the tungsten sputtering by sputtering of a low-Z material (boron-coated antennas) or by decreasing the image currents on the antenna limiters (broad-limiter or 3-strap antennas). The latter solution is the most promising one, because it reduces the image currents and the parallel electric field most significantly. When operating the 3-strap antenna with dipole phasing and a power ratio between the central and outer straps in the range (1.5, 2.0), tungsten sputtering at the antenna limiters is minimized [7]. Further experiments and simulations show that the RF voltages measured on the antenna limiters correlate with the measured tungsten effective sputtering yield and are consistent with the calculated local parallel electric field at the antenna limiters [18].

Following the previous achievement, EMC3-EIRENE simulations for ITER and DEMO are being carried out to find out optimized gas puff locations and gas puff rate for the best ICRF power coupling. Theoretical approaches [22-24] to model sheaths are being developed and will be validated against measurements on ASDEX Upgrade [8, 25] and on IShTAR [26, 27]. The combined theory and experiments will provide useful guidance on future directions for antenna designs with reduced sheaths and suppressed RF-specific impurities.

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